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Docket No. 50-320
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April 23, 1985

Docket No. 50-320

Mr. F. R. Standerfer
Vice President/Director
Three Mile Island Unit 2
GPU Nuclear Corporation
P.O. Box 480
Middletown, PA 17057

Dear Mr. Standerfer:

Subject: Three Mile Island Nuclear Station, Unit 2
Operating License DPR-73
Docket No. 50-520
Technical Specification Change Request No. 47

The Nuclear Regulatory Commission has issued the enclosed Amendment of Order in response to your February 15, 1985 request to modify sections of the Proposed Technical Specifications (PTS). Your proposal included requests for an increase in the minimum required boron concentration, the addition of water level and monitoring requirements in the Spent Fuel Storage Pool "A" and the Fuel Transfer Canal, the addition of load handling requirements for the Fuel Handling Building, minor editorial changes, and changes to the associated bases to reflect PTS modifications.

We have reviewed your safety evaluations supporting your proposal and conclude that the changes are acceptable with some modifications as discussed with your staff. PTS changes that are the subject of this letter will become effective May 31, 1985.

Since the February 11, 1980 Order imposing the PTS is currently pending before the Atomic Safety and Licensing Board, the staff will be advising the Licensing Board of this Amendment of Order through a Notice of Issuance of Amendment of Order and a Motion to Conform the Proposed Technical Specifications in Accordance Therewith.

OFFICE	TMIPO	TMIPO	TMI	ELD	NRR	NRR	
SURNAME	TPoindexter	R.Weller	BJSNYDER	W.D Paton	EISENHUT	DENTON	
DATE	4/3/85	4/12/85	4/15/85	4/4/85	4/ /85	4/22/85	

Federal Register Notices for the subject issuance are enclosed. Copies of the related Safety Evaluation and revised pages for the PTS are also enclosed.

Sincerely,

Original signed by
B. J. Snyder

Bernard J. Snyder, Program Director
Three Mile Island Program Office
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment of Order
2. Safety Evaluation
3. Proposed Technical Specification Page Changes
4. Notice of Environmental Assessment and Finding of No Significant Impact
5. Federal Register Notices

cc: T. F. Demmitt
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Enclosure 1

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter of	)	
GENERAL PUBLIC UTILITIES NUCLEAR CORPORATION	)	Docket No. 50-320
(Three Mile Island Nuclear Station, Unit 2)	)	

AMENDMENT OF ORDER

I.

GPU Nuclear Corporation, Metropolitan Edison Company, Jersey Central Power and Light Company and Pennsylvania Electric Company (collectively, the licensee) are the holders of Facility Operating License No. DPR-73, which had authorized operation of the Three Mile Island Nuclear Station, Unit 2 (TMI-2) at power levels up to 2772 megawatts thermal. The facility, which is located in Londonderry Township, Dauphin County, Pennsylvania, is a pressurized water reactor previously used for the commercial generation of electricity.

II.

By Order for Modification of License, dated July 20, 1979, the licensee's authority to operate the facility was suspended and the licensee's authority was limited to maintenance of the facility in the present shut-down cooling mode (44 Fed. Reg. 45271). By further Order of the Director, Office of Nuclear Reactor Regulation, dated February 11, 1980, a new set of formal license requirements was imposed to reflect the post-accident condition of the facility and to assure the continued maintenance of the current safe, stable, long-term cooling condition of the facility (45 Fed. Reg. 11282).

Although these requirements were imposed on the licensee by an Order of the Director of Nuclear Reactor Regulation, dated February 11, 1980, the TMI-2 license has not been formally amended. The requirements are reflected in the Recovery Mode Proposed Technical Specifications (PTS) presently pending before the Atomic Safety and Licensing Board. The revisions that are the subject of this order do not give the licensee authorizations that may be needed to undertake specific cleanup activities. These activities will require separate consideration by the staff per Section 6.8.2 of the PTS, individual staff safety evaluations and/or licensing actions as appropriate. Hereafter in this Amendment of Order, the requirements in question are identified by the applicable Proposed Technical Specification.

III.

By letter dated February 15, 1985, GPU Nuclear Corporation (GPUNC) proposed changes to the Proposed Technical Specifications (PTS) for Three Mile Island Unit 2.

The licensee proposed to increase the minimum boron concentration in the Reactor Coolant System (RCS) from 3500 ppm to 4350 ppm to insure that for any conceivable core configuration an adequate shutdown margin below criticality is maintained. The licensee also requested the application of minimum and maximum boron concentration limits to the Spent Fuel Storage Pool "A" (SFSPA) and the Fuel Transfer Canal (FTC) of 4350 and 6000 ppm, respectively. In case the water inventories of the RCS, the FTC

and the SFSPA communicated because of a leak or valve misalignment, the possibility of boron dilution of any of these water volumes would be minimized.

The licensee also proposed the addition to the PTS of water level and water level monitoring requirements for the SFSPA and FTC. These requirements would ensure adequate water shielding above fuel canisters. Specific levels will be stated in plant procedures approved by the NRC.

In addition, GPUNC proposed additional load handling requirements for the Fuel Handling Building that would preclude heavy load travel over a fuel canister unless a load drop analysis and associated procedures have been approved by the NRC. Per discussions with GPU, the staff added a requirement that the associated load drop Safety Evaluation be formally submitted to the NRC for approval.

The staff has also added per discussions with the licensee, definitions for Licensed Operator, Senior Licensed Operator and Fuel Handling Senior Reactor Operator.

The associated bases were also modified as requested by the licensee to reflect the above PTS changes.

Other changes proposed by the licensee were applicable to the Recovery Operations Plan (ROP) and are addressed in separate correspondence. Based on discussions herein and those in the attached Safety Evaluation, the staff concurs with the licensee's proposed changes. Minor modifications to the licensee's changes have been made by the staff and concurred with by the licensee as discussed above.

The staff's safety assessment of this matter as discussed above is set forth in the concurrently issued Safety Evaluation. Since the February 11, 1980 Order imposing the Proposed Technical Specifications is currently pending before the Atomic Safety and Licensing Board, the staff will be advising the Licensing Board of this Amendment of Order through a Notice of Issuance of Amendment of Order and a Motion to Conform Proposed Technical Specifications in Accordance Herewith.

It is further determined that the modification does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. In light of this determination and as reflected in the Environmental Assessment and Notice of Finding of No Significant Environmental Impact prepared pursuant to 10 CFR 51.2 and 51.30 through 51.32 issued concurrently herewith, it was concluded that the action is insignificant from the standpoint of environmental impact and that an environmental impact statement need not be prepared.

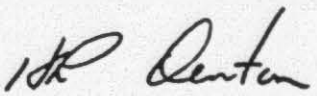
IV.

Accordingly, pursuant to the Atomic Energy Act of 1954, as amended, the Director's Order of February 11, 1980, is hereby revised to incorporate the deletions, additions, and modifications set forth in Enclosure 3 hereto. This Amendment of Order shall be effective on April 30, 1985.

For further details with respect to this action, see (1) Letter to B. J. Snyder, USNRC, from F. R. Standerfer, GPUNC, Technical Specification Change Request 47, Recovery Operations Plan Change Request 27 and (2) the Director's Order of February 11, 1980.

All the above documents are available for inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, DC, and at the Commission's Local Public Document Room at the State Library of Pennsylvania, Government Publications Section, Education Building, Commonwealth and Walnut Streets, Harrisburg, Pennsylvania 17126.

FOR THE NUCLEAR REGULATORY COMMISSION



Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Effective Date: May 31, 1985
Dated at Bethesda, Maryland
Issuance Date: April 23, 1985

Enclosure 2

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

GPU NUCLEAR CORPORATION

METROPOLITAN EDISON COMPANY

PENNSYLVANIA ELECTRIC COMPANY

JERSEY CENTRAL POWER AND LIGHT COMPANY

DOCKET NO. 50-320

THREE MILE ISLAND NUCLEAR STATION UNIT NO. 2

INTRODUCTION

By letter dated February 15, 1985, GPU Nuclear Corporation (GPUNC) requested the approval of changes to the Proposed Technical Specifications (PTS) of Operating License No. DPR-73 and provided supporting information for the proposed modifications. Also included was a request to modify the Recovery Operations Plan (ROP). Our review of the latter request is discussed in separate concurrently issued correspondence. The PTS changes were requested by the licensee to support upcoming recovery activities such as plenum removal and defueling. However, any PTS change approvals issued by the staff do not authorize such activities to occur. Approvals to conduct plenum removal or defueling will be addressed as separate staff actions.

DISCUSSION

Definition

Sections 1.15, 1.18, 1.19, 1.20

Per discussions with the licensee, the staff has added definitions for a Reactor Operator, Senior Reactor Operator and Fuel Handling Senior Reactor Operator. This information was added to define the different types of operators at TMI-2 and the licensing limitations of each.

LIMITING CONDITIONS FOR OPERATION

Section 3.1

The licensee requested that the lower boron concentration limit be revised from the present value of 3500 ppm to 4350 ppm. The staff had previously approved an increase in the lower boron limit from 3000 ppm to 3500 ppm in an Amendment of Order dated July 17, 1984. The 3500 ppm value was based on all credible core reconfigurations. However, the licensee has conservatively chosen to borate to a level that would ensure that the reactor coolant system (RCS) will remain subcritical for all conceivable core reconfigurations that could result from planned recovery operations. The staff had previously evaluated the licensee's criticality calculations submitted in a GPU letter dated November 8, 1984. We concurred with their analysis in a letter dated March 15, 1985, concluding that a shutdown margin of 1% ($K_{eff} \leq 0.99$) under any conceivable core configuration would be maintained at a 4350 ppm boron level.

The licensee also requested that the same 4350 ppm minimum and 6000 ppm maximum boron requirement be added for the Spent Fuel Storage Pool "A" and the Fuel Transfer Canal (deep end). These values are equal to those required in the RCS.

This will ensure that water inventories to be processed will not have a boron concentration less than 4350 ppm, thereby minimizing the effects of communication between the RCS and these inventories, should it occur.

The licensee has also requested an editorial change to Specification 3.1.1.1 to more clearly indicate their intent to maintain one decay heat pump operable at all times. This will insure sump recirculation capability until the functionally equivalent system is approved by the staff. Based on the above discussion, the staff concurs with the proposed changes to Specification 3.1.

Section 3.9

The licensee has proposed water levels and level monitoring requirements for Spent Fuel Pool "A" and the Fuel Transfer Canal in Specifications 3.9.1, 3.9.2, 3.9.3 and 3.9.4. For each area, two independent instruments will be required to be operable. With only one of the two water level monitoring instruments available, the licensee proposed a requirement to verify that water level is within its required limits and a second instrument be restored within 7 days. The staff, per discussions with the licensee, has modified the wording to state that should one of the independent level monitors become inoperable, an immediate verification of level should be made. Secondly, a re-verification of level should be made at least once per 24 hours thereafter. If both monitors are lost the staff, per discussions with the licensee, has modified the wording from requiring that one instrument be returned to operable status to requiring that at least

one instrument be returned to operable status. The re-wording recognizes the fact that it may be possible to restore two instruments to operable conditions.

The staff agrees with the licensee that the required water level values will vary depending on the amount of shielding required. Therefore, the specific values have not been included in the specifications. Instead, the licensee has proposed to submit Spent Fuel Pool "A" and Fuel Transfer Canal (deep end) water level procedures to the staff for approval. The NRC concurs with this method of establishing a water level value because of the potential for different levels being required depending on the operation in progress. A staff review of the values as they are developed will ensure that the health and safety of the workers and the public have not been adversely affected. Based on the above discussion, the staff concurs with proposed Specifications 3.9.1, 3.9.2, 3.9.3, and 3.9.4 with modifications as stated.

Section 3.10

The licensee has proposed to add load handling restrictions (Specification 3.10.2) for the Fuel Handling Building. This would preclude the travel of loads weighing in excess of 2400 pounds over canisters that conta[illegible] core material or any other areas in the Fuel Handling Building that have not been analyzed in an NRC approved load drop analysis. The staff concurs with this addition because it will minimize the likelihood of a load drop

accident onto a canister. The staff also concurs with the licensee's editorial modification of Specification 3.10.1 to clarify the Containment Building load handling requirements.

As stated by the licensee, it should be noted that the specification wording "approved by the NRC" is limited to "on-docket" approvals. Any Unit Work Instruction (UWI) or procedure whose load drop analysis is within the bounds of the "NRC approved analysis" need not have another load drop analysis re-approved. The exception would be if the bounds of the formal on-docket approval are exceeded. In the latter case, formal NRC approval is still required. The staff, per discussions with the licensee, has therefore added the requirement that a docketed load drop safety evaluation for carrying one canister over another shall be submitted for NRC approval. Based on the above discussion, the staff concurs with the modifications to Specification 3.10.

BASES

Bases sections 3/4.1.1, 3/4.9.1, 3/4.9.2, 3/4.9.3 and 3/4.9.4 have been added or modified as appropriate to reflect the above discussed LCO modifications.

ENVIRONMENTAL CONSIDERATIONS

We have determined that the changes do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, and, as reflected in the Environmental Assessment and Notice of Finding of

No Significant Environmental Impact prepared pursuant to 10 CFR 51.21 and 51.30 through 51.32, issued concurrently herewith, we have further concluded that the change involves an action which is insignificant from the standpoint of environmental impact and that an environmental impact statement need not be prepared in connection with the issuance of this action.

CONCLUSIONS

Based upon our review of the above discussed changes as modified, the staff finds that the requested revision of the proposed Technical Specifications is acceptable.

We have also concluded, based on the considerations discussed above, that:

(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and

(2) such activities will be conducted in compliance with the Commission's regulations and the implementation of this change will not be inimical to the common defense and security or to the health and safety of the public.

ENCLOSURE 3

FACILITY OPERATING LICENSE NOL DPR-73

DOCKET NO. 50-320

The following list of pages of the Appendix "A", Proposed Technical Specifications have been modified as a result of this Amendment of Order. Therefore, you should replace your present pages with those enclosed.

III
V
VI
VII
1-4
3.1.1
3.1-2
3.9-1
3.9-2
3.9-3
3.10-1
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B 3/4 1-1
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SURVEILLANCE REQUIREMENTS

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LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

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BASES

BASES

CORE ALTERATION

1.15 CORE ALTERATION shall be the movement or manipulation of any reactor component (including fuel) within the reactor pressure vessel with the vessel head removed and fuel in the vessel. Suspension of CORE ALTERATION shall not preclude completion of movement of a component to a safe conservative position.

LOSS-TO AMBIENT

1.16 LOSS-TO-AMBIENT is a passive cooling mode by which decay heat, generated by the reactor core, is removed and transferred to the surrounding environment by air and passive components (i.e., Reactor Vessel) inside the Reactor Building.

ACCIDENT GENERATED WATER

1.17 ACCIDENT GENERATED WATER, as defined in the settlement of the City of Lancaster litigation, is:

(a) Water that existed in the TMI-2 Auxiliary, Fuel Handling, and Containment Buildings including the primary system as of October 16,1979, with the exception of water which as a result of decontamination operations becomes commingled with non-accident generated water such that the commingled water has a tritium content of 0.025 µCi/ml or less before processing;

(b) Water that has a total activity of greater than one µCi/ml prior to processing except where such water is originally non-accident water and becomes contaminated by use in cleanup;

(c) Water that contains greater than 0.025 µCi/ml of tritium before processing.

1.18 LICENSED OPERATOR (OL) - any individual who possesses an NRC operator's license pursuant to Title 10, Code of Federal Regulations, Part 55, "Operators Licenses."

1.19 SENIOR LICENSED OPERATOR (SOL) - any individual who posseses an NRC Senior Operator's license pursuant to Title 10, Code of Federal Regulations, Part 55, "Operators Licenses."

1.20 FUEL HANDLING SENIOR LICENSED OPERATOR (SOL-FH) - an individual licensed by the Nuclear Regulatory Commission to supervise fuel handling and core alterations operations.

3.1 WATER INJECTION COOLING AND REACTIVITY CONTROL SYSTEMS

3.1.1 BORATION CONTROL

BORATED COOLING WATER INJECTION

3.1.1.1 At least two systems capable of injecting borated cooling water into the Reactor Coolant System shall be OPERABLE with:

a. One system comprised of the Standby Reactor Coolant System Pressure Control System.

b. The second system comprised of:

1. An OPERABLE flow path from the BWST (The BWST shall contain at least 100,000 gallons of borated water at a minimum temperature of 50°F and a boron concentration between 4350 and 6000 ppm) and

2. Two Mini Decay Heat Removal Pumps and heat exchangers and the associated flow path shall be OPERABLE, and
3. One Decay Heat Removal Pump and associated flow path shall be operable.

APPLICABILITY: RECOVERY MODE

ACTION:

a. With the Standby Reactor Coolant System Pressure Control System inoperable, suspend all operations involving CORE ALTERATIONS and/or the Reactor Coolant System and restore the inoperable system to OPERABLE status within 72 hours.

b. With one Mini Decay Heat Removal Pump or heat exchanger or the associated flowpath inoperable, restore to OPERABLE status within 72 hours.

c. With two Mini Decay Heat Removal Pumps or heat exchangers or the associated flowpath inoperable, immediately suspend all operations involving CORE ALTERATIONS and the Reactor Coolant System and restore both to Operable status within 72 hours.

d. With no Decay Heat Removal Pump or associated flowpath OPERABLE, immediately suspend all operations involving CORE ALTERATIONS and Reactor Coolant System and restore one Decay Heat Removal Pump and associated flowpath to OPERABLE status within 72 hours.

BORON CONCENTRATION

3.1.1.2 The boron concentration of the coolant in all filled portions of the Reactor Coolant System shall be maintained between 4350 and 6000 ppm and at a temperature above 50°F. 1

APPLICABILITY: RECOVERY MODE

ACTION:

If either of the above conditions are not satisfied (Boron Concentration between 4350 and 6000 ppm and temperature above 50°F) immediately suspend all activities involving CORE ALTERATION or the Reactor Coolant System and take action in accordance with procedures approved pursuant to Specification 6.8.2 to restore the concentration to within acceptable limits. 1

3.1.1.3 The boron concentration of the water in all filled portions of the Fuel Transfer Canal (deep end) and the Spent Fuel Storage Pool "A" shall be maintained between 4350 and 6000 ppm.

APPLICABILITY: RECOVERY MODE

ACTION

If the above condition is not satisfied (Boron Concentration between 4350 and 6000 ppm), take action necessary to restore the boron concentration to within acceptable limits.

3.1.3 CONTROL ASSEMBLIES

MECHANISMS

3.1.3.1 deleted.

3.9 RADIOACTIVE WASTE STORAGE

SPEND FUEL STORAGE POOL "A" WATER LEVEL MONITORING

3.9.1 Two independent Spent Fuel Storage Pool "A" water level monitoring instruments shall be OPERABLE.

APPLICABILITY: Whenever any Canister containing core material is in the Spent Fuel Storage Pool "A".

ACTION:

a. With only one Spent Fuel Storage Pool "A" water level monitoring instrument OPERABLE, immediately verify that the water level is within limits, re-verify the level at least once per 24 hours and restore a second instrument to OPERABLE status within 7 days.

b. With no Spent Fuel Storage Pool "A" water level monitoring instruments OPERABLE, terminate all activities involving any Canister containing core material in or over Spent Fuel Storage Pool "A" and all operations involving changes in Spent Fuel Storage Pool "A" water inventory and restore at least one instrument to OPERABLE status within 24 hours.

SPENT FUEL STORAGE POOL "A" WATER LEVEL

3.9.2 The water level in Spent Fuel Storage Pool "A" shall be maintained as specified per NRC approved procedures.

APPLICABILITY: Whenever any Canister containing core material is in the Spent Fuel Storage Pool "A".

ACTION:

With Spent Fuel Storage Pool "A" water level not in accordance with approved procedures, terminate all activities involving any Canister containing core material in or over Spent Fuel Storage Pool "A" and restore the water level to within specification within 24 hours.

FUEL TRANSFER CANAL (DEEP END) WATER LEVEL MONITORING

3.9.3 Two independent Fuel Transfer Canal (deep end) water level monitoring instruments shall be OPERABLE.

APPLICABILITY: Whenever any Canister containing core material and/or the plenum assembly is in the Fuel Transfer Canal (deep end).

ACTION:

a. With only one Fuel Transfer Canal (deep end) water level monitoring instrument OPERABLE, immediately verify that the water level is within limits, re-verify the level at least once per 24 hours and restore a second instrument to OPERABLE status within 7 days.

b. With no Fuel Transfer Canal (deep end) water level instruments OPERABLE, terminate all activities involving any Canister containing core material in or over the Fuel Transfer Canal (deep end) and/or all activities involving the plenum assembly and all operations involving changes in the Fuel Transfer Canal (deep end) water inventory and restore one inoperable instrument to OPERABLE status within 24 hours.

FUEL TRANSFER CANAL (DEEP END) WATER LEVEL

3.9.4 The water level in the Fuel Transfer Canal (deep end) shall be maintained at the level specified per NRC approved procedures.

APPLICABILITY: Whenever any Canister containing core material and/or the plenum assembly is in the Fuel Trnasfer Canal (deep end).

ACTION:

With the Fuel Transfer Canal (deep end) water level not in accordance with approved procedures, terminate all activities involving any Canister containing core material in or over the Fuel Transfer Canal (deep end) and/or all activities involving the plenum assembly and restore the water level to within specification within 24 hours.

FUEL HANDLING BUILDING/AUXILIARY BUILDING AIR CLEANUP SYSTEMS

3.9.12.1 The Fuel Handling Building Air Cleanup Exhaust System shall be OPERABLE with exhaust ventilation flow through the HEPA filters during system operation. The Fuel Handling Building Air Cleanup Exhaust System is OPERABLE when two of the four system air cleanup exhaust fans are OPERABLE.

APPLICABILITY: RECOVERY MODE

ACTION:

a. With the Fuel Handling Building Air Cleanup Exhaust System inoperable due to flow requirements, return the flow to within acceptable limits within four (4) hours or;

b. With the Fuel Handling Building Air Cleanup Exhaust System inoperable (other than as allowed in paragraph 3.9.12.1 a above), suspend all operations involving movement of liquid and gaseous radioactive wastes in the Fuel Handling Building (other than sampling evolutions required by the Technical Specifications or RECOVERY OPERATIONS PLAN) until the system is restored to OPERABLE status.

3.9.12.2 The Auxiliary Building Air Cleanup Exhaust System shall be OPERABLE with exhaust ventilation flow through the HEPA filters during system operation. The Auxiliary Building Air Cleanup Exhaust System is OPERABLE when two of the four system air cleanup exhaust fans are OPERABLE.

APPLICABILITY: RECOVERY MODE

ACTION:

a. With the Auxiliary Building Air Cleanup Exhaust System inoperable due to flow requirements, return the flow to within acceptable limits within four (4) hours or;

b. With the Auxiliary Building Air Cleanup Exhaust System inoperable (other than as allowed in paragraph 3.9.12.2 a above), suspend all operations involving movement of liquid and gaseous radioactive wastes in the Auxiliary Building (other than sampling evolutions required by the Technical Specifications or RECOVERY OPERATIONS PLAN) until the system is restored to OPERABLE status.

ACCIDENT GENERATED WATER

3.9.13 Discharge of ACCIDENT GENERATED WATER shall be prohibited until approved by the NRC. ACCIDENT GENERATED WATER shall be discharged in accordance with procedures approved pursuant to Specification 6.8.2.

APPLICABILITY: RECOVERY MODE

ACTION:

None except as provided in Specification 3.0.3.

REACTOR BUILDING SUMP WATER

3.9.14 Deleted.

3.10 CRANE OPERATIONS

CONTAINMENT BUILDING

3.10.1 Loads in excess of 2400 lbs. shall be prohibited from travel over the following areas unless a docketed Safety Evaluation for the activity is approved by the NRC:

a. reactor vessel

b. incore instrument seal table and guide tubes (includes travel by polar crane block)

c. deep end of the Fuel Transfer Canal

d. any Canister that contains core material regardless of its location

e. areas not previously analyzed in a docketed, NRC approved load drop analysis

APPLICABILITY: RECOVERY MODE

ACTION:

With the requirements of the above specification not satisfied, place the crane load in a safe condition and correct the circumstances which caused or allowed the Limiting Condition for Operations to be exceeded prior to continuing crane operations limited by Specification 3.10.1. Prepare and submit a special report to the Commission pursuant to Specification 6.9.2 within the next 30 days.

FUEL HANDLING BUILDING

3.10.2 Loads in excess of 2400 lbs. shall be probibited from travel over the following areas unless a docketed Safety Evaluation for the activity is approved by the NRC:

a. any area of the spent fuel storage pool which contains one or more Canisters which contain core material

b. any Canister that contains core material, regardless of its location

c. areas not previously analyzed in a docketed, NRC approved load drop analysis

APPLICABILITY: RECOVERY MODE

ACTION:

With the requirements of the above specification not satisfied, place the crane load in a safe condition and correct the circumstances which caused or allowed the Limiting Condition for Operations to be exceeded prior to continuing crane operations limited by Specification 3.10.2. Prepare and submit a special report to the Commission pursuant to Specification 6.9.2 within the next 30 days.

3.4.1 WATER INJECTION COOLING AND REACTIVITY CONTROL SYSTEMS

BASES

3/4.1.1 BORATION CONTROL

The limitation on minimum boron concentration ensures that the core will remain subcritical under all credible conditions which may exist during the long-term cooling mode. The maximum boron concentration is provided to ensure that precipitation of boron will not occur in the RCS and thereby cause possible flow restrictions. The specification requiring the OPERABILITY of two systems capable of injecting borated cooling water into the RCS within the required boron concentration limits. The required volume of borated water in the BWST provides sufficient water to keep the core covered in the event of an unisolatable leak from the reactor vessel. The specified water volume is sufficient to provide a continuous supply of water to the vessel during the interim period before a sump recirculating flow path (e.g., one Decay Heat Removal Pump and associated flowpath) can be placed in service. Minimum boron concentration limits have been provided for the Refueling Canal (deep end) and Spent Fuel Storage Pool "A" to provide assurance that any event involving these volumes of water will not result in a margin of safety less than that analyzed for the reactor vessel. Requirements for operability of the Mini Decay Heat Removal System or an operable decay heat removal pump are stated in order to provide for injection of borated cooling water to the RCS from the BWST.

3/4.1.3 CONTROL ASSEMBLIES

All full-length control rods were fully inserted as a result of the reactor trip on March 28, 1979. This Specification has been deleted since the reactor vessel head has been removed.

3/4.9 RADIOACTIVE WASTE STORAGE

BASES

3/4.9.1 SPENT FUEL STORAGE POOL "A" WATER LEVEL MONITORING

Spent Fuel Storage Pool Water "A" Level Monitoring instrumentation has been provided to assure the capability to monitor water level in the Spent Fuel Storage Pool "A".

3/4.9.2 SPENT FUEL STORAGE POOL "A" WATER LEVEL

The water level in the Spent Fuel Storage Pool "A" has been established to limit the dose rate, due to the storage of Canisters, to acceptable levels.

3/4.9.3 FUEL TRANSFER CANAL (DEEP END) WATER LEVEL MONITORING

Fuel Transfer Canal Water Level Monitoring instrumentation has been provided to assure the capability to monitor water level in the deep end of the Fuel Transfer Canal.

3/4.9.4 FUEL TRANSFER CANAL (DEEP END) WATER LEVEL:

The water level in the Fuel Transfer Canal (deep end) has been established to limit the dose rate, due to the storage of the plenum assembly and Canisters, to acceptable levels.

3/4.9.12 FUEL HANDLING BUILDING/AUXILIARY BUILDING AIR CLEANUP SYSTEMS

The requirements for the Fuel Handling/Auxiliary Building Air Cleanup Systems to be operating or OPERABLE ensure that all radioactive material released from the liquid radioactive wastes being stored in the new radwaste storage tanks which have been installed in the spent fuel storage pool or elsewhere in the Auxiliary Building will be filtered through the HEPA filters prior to release to the atmosphere.

3/4.9.13 ACCIDENT GENERATED WATER

These specifications are provided to ensure compliance with the Commission's Statement of May 25, 1979, and the Commission's Statement of Policy and Notice of Intent to Prepare a Programmatic Environmental Impact Statement of November 21, 1979, which prohibit these actions pending evaluation of the environmental impacts of such actions. The PEIS issued in March, 1981, deferred a decision on the ultimate disposal of processed water. Further Commission action is necessary prior to release of ACCIDENT GENERATED WATER.

However, the Commission has recognized that there may be emergency situations, not at this time forseen, which could require rapid action. In these situations, the Commission has indicated its intention to consult with the Council on Environmental Quality to the extent practicable.

3/4.9 RADIOACTIVE WASTE STORAGE

BASES

ACCIDENT GENERATED WATER, as defined in the settlement of the City of Lancaster litigation, is:

(a) Water that existed in the TMI-2 Auxiliary, Fuel Handling, and Containment Buildings including the primary system as of October 16, 1979, with exception of water wich as a result of decontamination operations becomes commingled with non-accident generated water such that the commingled water has a tritium content of 0.025 μCi/ml or less before processing;

(b) Water that has a total activity of greater than one μCi/ml prior to processing except where such water is originally non-accident water and becomes contaminated by use in cleanup;

(c) Water that contains greater than 0.025 μCi/ml of tritium before processing.

3/4.9.14 Deleted.

Enclosure 4

UNITED STATES NUCLEAR REGULATORY COMMISSION

GENERAL PUBLIC UTILITIES NUCLEAR CORPORATION

DOCKET NO. 50-320

ENVIRONMENTAL ASSESSMENT AND NOTICE OF FINDING

OF NO SIGNIFICANT ENVIRONMENTAL IMPACT

The U.S. Nuclear Regulatory Commission (the Commission) is planning to issue an Amendment of Order to Facility Operating License No. DPR-73, issued to General Public Utilities Nuclear Corporation (the licensee), for operation of the Three Mile Island Nuclear Station, Unit 2 (TMI-2), located in Londonderry Township, Dauphin County, Pennsylvania.

ENVIRONMENTAL ASSESSMENT

Identification of Proposed Action: The action being considered by the Commission is an Amendment of the Director of Nuclear Reactor Regulation's Order dated February 11, 1980.

This Amendment of Order is being issued in response to General Public Utilities Nuclear Corporation's (GPUNC) letter dated February 15, 1985.

The Need for the Action: The Amendment of Order is warranted because of the need to modify the Proposed Technical Specifications (PTS) for future recovery operations at TMI-2. The types of actions to be taken include an increase in the minimum required boron concentration in the RCS; the addition of boron concentration requirements in the Spent Fuel Storage Pool "A" and the Fuel Transfer Canal; the addition of water level and water level monitoring requirements in the Spent Fuel Storage Pool "A" and the

Fuel Transfer Canal; and limiting heavy load travel over canisters containing core material.

Environmental Impacts of the Proposed Actions: The staff has evaluated the PTS modifications proposed by the Amendment of Order and concluded that it will not result in significant increases in airborne or liquid radioactivity inside the reactor building or in corresponding releases to the environment. There are also no non-radiological impacts to the environment as a result of these actions.

Alternative to this Action: Since we have concluded that there is no significant environmental impact associated with the subject Amendment of Order, any alternatives to these changes will have either no significant environmental impact or greater environmental impact. The principal alternative would be to deny the requested actions. This would not reduce significant environmental impacts of plant operations and would result in the application of overly restrictive regulatory requirements when considering the unique conditions at TMI-2.

Agencies and Persons Consulted: The NRC staff reviewed the licensee's request and did not consult other agencies or persons.

Alternate Use of Resources: This action does not involve the use of resources not previously considered in connection with the Final Programmatic Impact Statement for TMI-2 dated March 1981.

Finding of No Significant Impact: The Commission has determined not to prepare an environmental impact statement for the subject Amendment of Order. Based upon the foregoing environmental assessment, we conclude that this action will not have a significant effect on the quality of the human environment.

For further details with respect to this action see (1) Letter to B. J. Snyder, USNRC, from F. R. Standerfer, GPUNC, Technical Specification Change Request No. 47, dated February 15, 1985; and (2) the Director's Order of February 11, 1980.

All of the above documents are available for inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, DC, and at the Commission's Local Public Document Room at the State Library of Pennsylvania, Government Publications Section, Education Building, Commonwealth and Walnut Streets, Harrisburg, Pennsylvania 17126.

FOR THE NUCLEAR REGULATORY COMMISSION

Bernard J. Snyder

Bernard J. Snyder, Program Director
Three Mile Island Program Office
Office of Nuclear Reactor Regulation

☆ U.S. GOVERNMENT PRINTING OFFICE: 1978—633-712

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

April 23, 1985

Docket No. 50-320

Docketing and Service Section
Office of the Secretary of the Commission

SUBJECT: Three Mile Island Nuclear Station, Unit 2
Operating License No. DPR-73; Docket No. 50-320
Amendment of Order

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies ( ) of the Notice are enclosed for your use.

☐ Notice of Receipt of Application for Construction Permit(s) and Operating License(s).

☐ Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.

☐ Notice of Availability of Applicant's Environmental Report.

☐ Notice of Proposed Issuance of Amendment to Facility Operating License.

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☐ Notice of Availability of NRC Draft/Final Environmental Statement.

☐ Notice of Limited Work Authorization.

☐ Notice of Availability of Safety Evaluation Report.

☐ Notice of Issuance of Construction Permit(s).

☐ Notice of Issuance of Facility Operating License(s) or Amendment(s).

☒ Other: Amendment of Order

Enclosure:
As Stated

Office of Nuclear Reactor Regulation

Bernard J. Snyder, Program Director
Three Mile Island Program Office

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter of	)	
GENERAL PUBLIC UTILITIES NUCLEAR CORPORATION	)	Docket No. 50-320
(Three Mile Island Nuclear Station, Unit 2)	)	

AMENDMENT OF ORDER

I.

GPU Nuclear Corporation, Metropolitan Edison Company, Jersey Central Power and Light Company and Pennsylvania Electric Company (collectively, the licensee) are the holders of Facility Operating License No. DPR-73, which had authorized operation of the Three Mile Island Nuclear Station, Unit 2 (TMI-2) at power levels up to 2772 megawatts thermal. The facility, which is located in Londonderry Township, Dauphin County, Pennsylvania, is a pressurized water reactor previously used for the commercial generation of electricity.

II.

By Order for Modification of License, dated July 20, 1979, the licensee's authority to operate the facility was suspended and the licensee's authority was limited to maintenance of the facility in the present shutdown cooling mode (44 Fed. Reg. 45271). By further Order of the Director, Office of Nuclear Reactor Regulation, dated February 11, 1980, a new set of formal license requirements was imposed to reflect the post-accident condition of the facility and to assure the continued maintenance of the current safe, stable, long-term cooling condition of the facility (45 Fed. Reg. 11282).

Although these requirements were imposed on the licensee by an Order of the Director of Nuclear Reactor Regulation, dated February 11, 1980, the TMI-2 license has not been formally amended. The requirements are reflected in the Recovery Mode Proposed Technical Specifications (PTS) presently pending before the Atomic Safety and Licensing Board. The revisions that are the subject of this order do not give the licensee authorizations that may be needed to undertake specific cleanup activities. These activities will require separate consideration by the staff per Section 6.8.2 of the PTS, individual staff safety evaluations and/or licensing actions as appropriate. Hereafter in this Amendment of Order, the requirements in question are identified by the applicable Proposed Technical Specification.

III.

By letter dated February 15, 1985, GPU Nuclear Corporation (GPUNC) proposed changes to the Proposed Technical Specifications (PTS) for Three Mile Island Unit 2.

The licensee proposed to increase the minimum boron concentration in the Reactor Coolant System (RCS) from 3500 ppm to 4350 ppm to insure that for any conceivable core configuration an adequate shutdown margin below criticality is maintained. The licensee also requested the application of minimum and maximum boron concentration limits to the Spent Fuel Storage Pool "A" (SFSPA) and the Fuel Transfer Canal (FTC) of 4350 and 6000 ppm, respectively. In case the water inventories of the RCS, the FTC

and the SFSPA communicated because of a leak or valve misalignment, the possibility of boron dilution of any of these water volumes would be minimized.

The licensee also proposed the addition to the PTS of water level and water level monitoring requirements for the SFSPA and FTC. These requirements would ensure adequate water shielding above fuel canisters. Specific levels will be stated in plant procedures approved by the NRC.

In addition, GPUNC proposed additional load handling requirements for the Fuel Handling Building that would preclude heavy load travel over a fuel canister unless a load drop analysis and associated procedures have been approved by the NRC. Per discussions with GPU, the staff added a requirement that the associated load drop Safety Evaluation be formally submitted to the NRC for approval.

The staff has also added per discussions with the licensee, definitions for Licensed Operator, Senior Licensed Operator and Fuel Handling Senior Reactor Operator.

The associated bases were also modified as requested by the licensee to reflect the above PTS changes.

Other changes proposed by the licensee were applicable to the Recovery Operations Plan (ROP) and are addressed in separate correspondence. Based on discussions herein and those in the attached Safety Evaluation, the staff concurs with the licensee's proposed changes. Minor modifications to the licensee's changes have been made by the staff and concurred with by the licensee as discussed above.

The staff's safety assessment of this matter as discussed above is set forth in the concurrently issued Safety Evaluation. Since the February 11, 1980 Order imposing the Proposed Technical Specifications is currently pending before the Atomic Safety and Licensing Board, the staff will be advising the Licensing Board of this Amendment of Order through a Notice of Issuance of Amendment of Order and a Motion to Conform Proposed Technical Specifications in Accordance Herewith.

It is further determined that the modification does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. In light of this determination and as reflected in the Environmental Assessment and Notice of Finding of No Significant Environmental Impact prepared pursuant to 10 CFR 51.2 and 51.30 through 51.32 issued concurrently herewith, it was concluded that the action is insignificant from the standpoint of environmental impact and that an environmental impact statement need not be prepared.

IV.

Accordingly, pursuant to the Atomic Energy Act of 1954, as amended, the Director's Order of February 11, 1980, is hereby revised to incorporate the deletions, additions, and modifications set forth in Enclosure 3 hereto. This Amendment of Order shall be effective on April 30, 1985.

For further details with respect to this action, see (1) Letter to B. J. Snyder, USNRC, from F. R. Standerfer, GPUNC, Technical Specification Change Request 47, Recovery Operations Plan Change Request 27 and (2) the Director's Order of February 11, 1980.

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FOR THE NUCLEAR REGULATORY COMMISSION

Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Effective Date: May 31, 1985
Dated at Bethesda, Maryland
Issuance Date: April 23, 1985

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

April 23, 1985

Docket No. 50-320

Docketing and Service Section
Office of the Secretary of the Commission

SUBJECT: Three Mile Island Nuclear Station, Unit 2
Operating License No. DPR-73; Docket No. 50-320
Environmental Assessment and Notice of Finding of No Significant Environmental Impact

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☐ Notice of Issuance of Facility Operating License(s) or Amendment(s).

☒ Other: Notice of Finding of No Significant Environmental Impact

Enclosure:
As Stated

Office of Nuclear Reactor Regulation

Bernard J. Snyder

Bernard J. Snyder, Program Director
Three Mile Island Program Office

NRC FORM 102
(1-76)

UNITED STATES NUCLEAR REGULATORY COMMISSION

GENERAL PUBLIC UTILITIES NUCLEAR CORPORATION

DOCKET NO. 50-320

ENVIRONMENTAL ASSESSMENT AND NOTICE OF FINDING

OF NO SIGNIFICANT ENVIRONMENTAL IMPACT

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ENVIRONMENTAL ASSESSMENT

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Fuel Transfer Canal; and limiting heavy load travel over canisters containing core material.

Environmental Impacts of the Proposed Actions: The staff has evaluated the PTS modifications proposed by the Amendment of Order and concluded that it will not result in significant increases in airborne or liquid radioactivity inside the reactor building or in corresponding releases to the environment. There are also no non-radiological impacts to the environment as a result of these actions.

Alternative to this Action: Since we have concluded that there is no significant environmental impact associated with the subject Amendment of Order, any alternatives to these changes will have either no significant environmental impact or greater environmental impact. The principal alternative would be to deny the requested actions. This would not reduce significant environmental impacts of plant operations and would result in the application of overly restrictive regulatory requirements when considering the unique conditions at TMI-2.

Agencies and Persons Consulted: The NRC staff reviewed the licensee's request and did not consult other agencies or persons.

Alternate Use of Resources: This action does not involve the use of resources not previously considered in connection with the Final Programmatic Impact Statement for TMI-2 dated March 1981.

Finding of No Significant Impact: The Commission has determined not to prepare an environmental impact statement for the subject Amendment of Order. Based upon the foregoing environmental assessment, we conclude that this action will not have a significant effect on the quality of the human environment.

For further details with respect to this action see (1) Letter to B. J. Snyder, USNRC, from F. R. Standerfer, GPUNC, Technical Specification Change Request No. 47, dated February 15, 1985; and (2) the Director's Order of February 11, 1980.

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FOR THE NUCLEAR REGULATORY COMMISSION

Bernard J. Snyder, Program Director
Three Mile Island Program Office
Office of Nuclear Reactor Regulation